

APPROVED BY

Director of Nuclear Science & Engineering School Olég Yu. Dolmatov 2020

Course Name: Thermal Hydraulics in Nuclear Reactors

Field of Study: Nuclear Science and Technology

Programme name: Nuclear Science and Technology

Specialization: Nuclear Power Engineering

Level of Study: Master Degree Programme

Year of admission: 2019

Semester, year: semester 1, year 1

ECTS: 4

Total Hours: 144

Contact Hours: 48

• Lectures: 24

Practical experience: 24

Self-study: 96

Assessment: Credit-test

Division: Nuclear Fuel Cycle

Director of Programme Professor

Vera V. Verkhoturova Alexander G. Korotkikh



Course name: Thermal Hydraulics in Nuclear Reactors

Course Overview

| Course Objectives | The objective of the course is to form a set of competences (learning outcomes) |
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| | which will enable graduates to carry out their professional activity in the field of |
| | thermal physics and operation of nuclear reactors at nuclear power plants. |
| | Upon completion of the course, a graduate will obtain the knowledge of: |
| | - basics of structuring a report and preparing presentations using foreign |
| | language, accepted in the international environmentx |
| | - goals and objectives of scientific research in the field of professional |
| | activity, basic principles and methods of their organization; |
| | thermohydraulic calculations of nuclear power plants; |
| | heat exchange equipment using modern methods; |
| | calculation programs and information sources, |
| Learning Outcomes | Upon completion of the course, graduates are also expected to develop the |
| | following skills: |
| | - to develop a project taking into account the analysis of alternative options |
| | for its implementation, determine the target stages, the main directions of |
| | work; |
| | - to explain goals and formulate tasks related to the preparation and |
| | implementation of the project, determine the main stages and directions of |
| | work; |
| | - to compile and present technical and scientific information used in |
| | professional activities in the form of a presentation; |
| | to perceive authentic audio and video materials related to training direction; |
| | |
| | to compose a general plan of work on a given topic, suggest research methods and methods of result processing; |
| | - to perform thermohydraulic calculations of nuclear power plants, heat |
| | exchange equipment using modern methods; |
| | to use calculation programs and information sources; |
| | - to search and analyze information related to structures and operating |
| | experience of nuclear power plants, fuel elements, structural materials |
| | using scientific citation databases. |
| | Upon completion of the course, graduates should acquire the practical |
| | experience in: |
| | applying methods of development and project management; |
| | - applying methods for assessing resource requirements and project |
| | effectiveness; |
| | - applying skills of monologue speech in a foreign language according to the |
| | profile of specialty, reasonably expressing his position and using auxiliary |
| | means (tables, graphs, charts, etc.); |
| | applying acquired knowledge of a foreign language at a sufficient level in his future professional activities; |
| | applying systematic knowledge in the field of future professional activity; |
| | apprying systematic knowledge in the field of future professional activity, |

| | - applying in-depth knowledge on the chosen direction of training, basic |
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| | skills for conducting research on the proposed topic; |
| | - performing thermohydraulic calculations of nuclear power plants, heat |
| | exchange equipment using modern methods. |
| | The target course is taught using a variety of teaching forms such as: |
| | - 12 lectures; |
| | – 12 practical experiences; |
| | - 3 individual homework assignments; |
| | - 2 reviews: |
| | |
| | -2 tests; |
| | - 1 colloquium. |
| | The course consists of four sections, which are given below. |
| | Section 1. Heat release in a nuclear reactor. |
| | Section 2. Thermal conductivity in a nuclear reactor. |
| | Section 3. Convective heat transfer in a nuclear reactor. |
| | Section 4. Heat transfer by radiation in a gas cooled nuclear reactor. |
| | Each section includes several lectures and practical experiences. |
| | During the course, students perform practical tasks and 2 tests. The course ends |
| Course | with a colloquium. |
| Outline | Learners'self-study is arranged in a form of individual research of the topics and |
| | individual homework assignments. During the course of study, learners are |
| | expected to complete three individual homework assignments and two reviews. |
| | Individual homework assignment is a set of tasks each containing unique set |
| | parameters. It is obligatory for each student to present the results of individual |
| | homework assignment completion in a form of a report. The report must have a |
| | title page, initial data, task solution, conclusions, and final statement. The report |
| | must be defended in a class. This suggests students answering from 3 to 5 |
| | questions related to the topic of the assignment. |
| | <i>Review</i> is given to the student by the teacher with an indication of the databases |
| | and the deadlines for the assignment. The topic of review must be thoroughly |
| | |
| | researched. The materials of the review work must be presented in paper. Review |
| | includes the literature overview on the given topic and shall have the following |
| | parts: a title page, outline, introduction, main body sections, conclusion and |
| | reference list. |
| | The content of the course covers four main sections. Each section is studied |
| | through lectures and practical experiences. |
| | Section 1. Heat release in a nuclear reactor |
| | Lecture 1. The nuclear reactors. Processes of heat release in the core of nuclear |
| | reactor. |
| | Lecture 2. The distribution of the heat release in a nuclear reactor. |
| | Practical experience 1. Designs and characteristics of nuclear reactors. |
| Course | Practical experience 2. Calculation of the heat generated in the core of nuclear |
| Structure | reactor. |
| | Section 2. Thermal conductivity in a nuclear reactor. |
| | Lecture 3. Conductive heat flux. Fourier law. Thermal conductivity coefficient. |
| | Lecture 4. Steady-state conduction in a nuclear reactor design. |
| | Lecture 5. Heat conduction with the heat release in a fuel rod. |
| | Lecture 6. Unsteady heat conduction during cooling (heating) of body. |
| | Practical experience 3. Calculation of the heat flux and the temperature |
| | distribution in bodies of various shapes. |
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| | Practical experience 4. Calculation of the heat flux on the surface of the nuclear |
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| | reactor design. |
| | e |
| | Practical experience 5. Calculation of the heat flux and the temperature distribution in the fuel element. |
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| | Practical experience 6. Calculation of the heat release during shutdown of a |
| | nuclear reactor and temperatures in structures during their cooling (heating). |
| | Section 3. Convective heat transfer in a nuclear reactor. |
| | Lecture 7. Newton-Richman Law. Similarity and modeling of the convective heat |
| | transfer. |
| | Lecture 8. Heat transfer in a single-phase medium with free and forced flow of the |
| | coolant. |
| | Lecture 9. Heat transfer at the forced longitudinal and cross-washing tubes, tube |
| | and rod bundles. Heat transfer of liquid metal coolant. |
| | Lecture 10. Heat transfer during boiling and condensation. Critical heat flux. Lecture 11. The principle of thermohydraulic calculation of the nuclear reactor |
| | core cooled by single-phase and two-phase flows. |
| | Practical experience 7. Calculation of heat transfer in natural circulation of the |
| | coolant. |
| | Practical experience 8. Calculation of heat transfer in forced circulation of the |
| | coolant. |
| | Practical experience 9. Calculation of heat transfer in the forced longitudinal and |
| | cross-washing tubes, tube and rod bundles. |
| | Practical experience 10. Calculation of heat transfer during boiling liquid, heat |
| | transfer crisis in fuel assemblies. |
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| | Practical experience 11. Thermohydraulic calculation of the nuclear reactor core. |
| | Thermal calculation of heat exchange equipment. |
| | Section 4. Heat transfer by radiation in a gas cooled nuclear reactor. Lecture 12. Heat transfer by radiation in a gas cooled nuclear reactor. |
| | Practical experience 12. Calculation of complex heat transfer in power plants. |
| Facilities and | 1. Lecture hall with multimedia equipment and computers: Tomsk, Lenin ave. |
| Equipment | 30a, build. 4, room 31. |
| Equipment | In accordance with TPU rating system we use: |
| | - Current assessment which is performed on a regular basis during the |
| | semester by scoring the quality of mastering of theoretical material and the |
| Crading | results of practical activities (performance control questions, practical |
| Grading Policy | problems). Max score for current assessment is 100 points, min -55 |
| roncy | pionents). Max score for current assessment is foo points, min – 55 points. |
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| | - Course final assessment (credit test) is performed at the end of the |
| Course Deliev | semester according to the results of the current rating. |
| Course Policy | Attendance at lectures and practical experience are compulsory. |
| Teaching | Compulsory reading: |
| Aids and | 1. Прибытков И. А. Thermophysics = Теплофизика : учебное пособие / И. А. |
| Resources | Прибытков. — Москва : МИСИС, 2019. — 97 с. — Текст : электронный // |
| | Лань : электронно-библиотечная система. — URL: |
| | <u>https://e.lanbook.com/book/129050</u> (дата обращения: 17.12.2020). — Режим |
| | доступа: из корпоративной сети ТПУ. |
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| | Additional reading: |
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| | доступа: https://ezproxy.ha.tpu.ru:2056/book/9780128129081/combined-heat- and-power. — Загл. с экрана. |
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